Reactions in the new scheme

D. Brown, BNL



a passion for discovery



I am trying to capture your view of how to arrange nuclear data

- I will present the consensus view of required arrangement of nuclear data,
 - presented at SG38 Meeting in Tokai, Japan, Dec. 2013
 - revised and presented at SG38 Meeting in Paris, France, Apr. 2014
 - revised again for this meeting
- Issues dealt with already (I think)
 - Reaction major vs. energy major
 - Documentation requirements
- Element & attribute names are illustrative. They can be changed.

Requirements for a top level hierarchy for a next generation nuclear data format

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All MF's are covered

- 1 (general info.): <documentation>
- 2 (RR): <resonances>
- 3 (cross sections): <crossSection>
- 4 (n angle dist.): a <distribution>
- 5 (n energy dist.): a <distribution>
- 6 (energy-angle dists.): a <distribution> (note: <dcrossSection_dOmega> for LACS)
- 8 (FPY): a <distribution>

- 8 (decay): in
 <localMaterialDatabase>
- 9 (rad. decay mult.): a <distribution>
- 10 (prod. cross sections): a derived <crossSection>
- 12 (gamma yields): continuum gammas are in <distribution> or discretes in <localMaterialDatabase>
- 13 (gamma prod. cross sections): a derived transport data
- 14 (g angle dist.): a <distribution>
- 15 (g energy dist.): a <distribution>

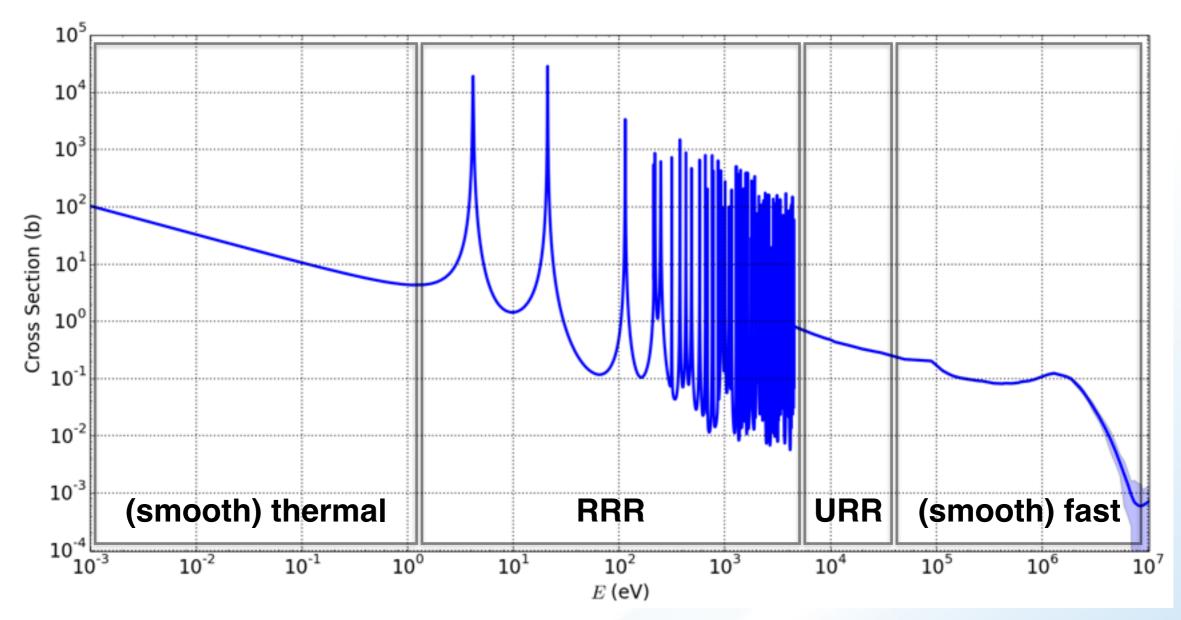


All MF's are covered

- 26 (g-, e- dists.): a <distribution>
- 27 (atomic scatt.): <dcrossSection_dOmega>
- 28 (atomic relax.): in
 <localMaterialDatabase>
- 30 (model param. cov.): a <covariance> & <sensitivity>
- 31 (nubar cov.): a <covariance>
- 32 (RR cov.): a <covariance> & <sensitivity>
- 33 (cross section cov.): a <covariance>
- 34 (ang. dist. cov): a
 covariance>

- 35 (energy dist. cov.): a <covariance>
- 40 (rad. prod. cov.): a <covariance>

This is the most-typical use case: a neutron hitting a nucleus



- With a little planning, we can also accommodate charged particle and gamma induced reactions too
- TSL data is actually not a stretch

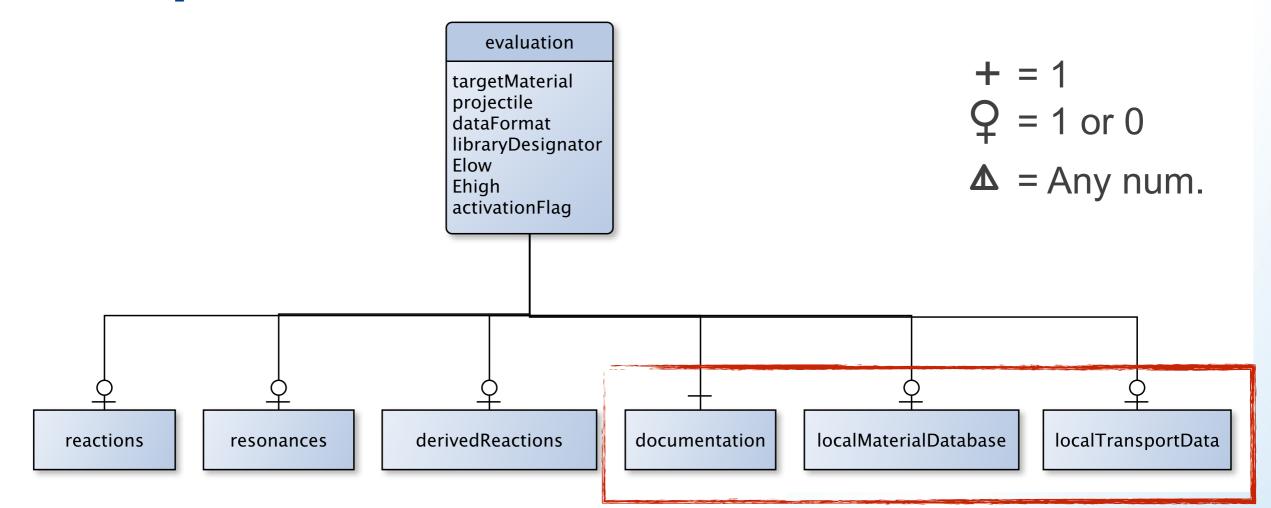


The top level: an <evaluation>

evaluation One temperature only; targetMaterial projectile Q = 1 or 0one projectile; dataFormat libraryDesignator Elow one target material Δ = Any num. Ehigh activationFlag derivedReactions localMaterialDatabase localTransportData reactions resonances documentation



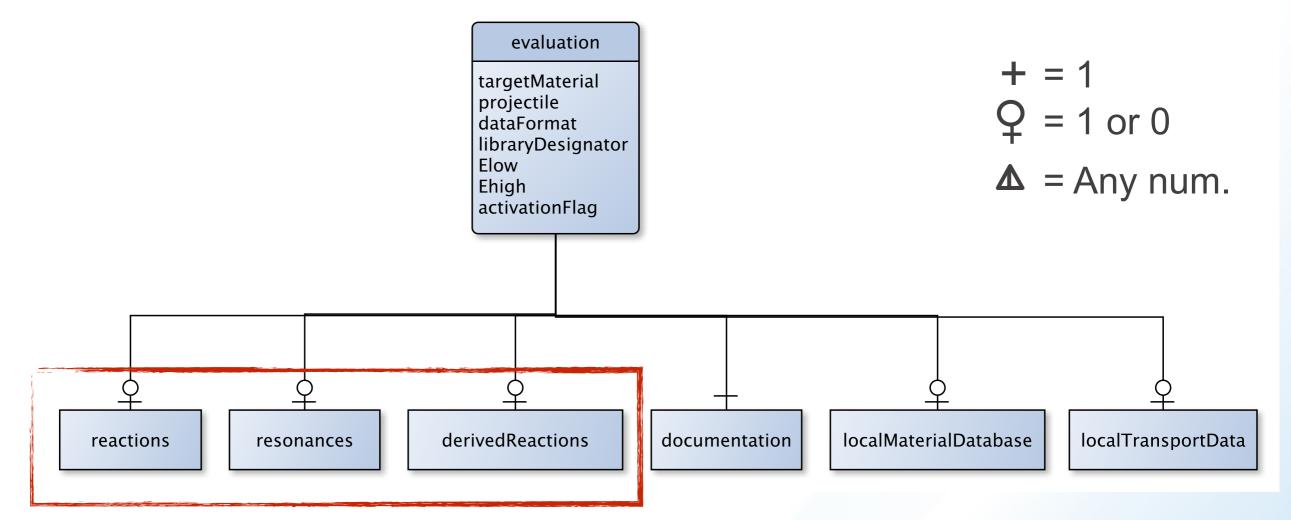
The top level: an <evaluation>, cont.



- <localMaterialDatabase> Fulfills need to override default material information (esp. for older evaluations) will discuss later this morning
- <localTransportData > Data needed for transport, overriding or describing things specific for this evaluation (e.g. local group structure) will discuss this tomorrow afternoon
- <documentation> Allowed at all levels thoroughly discussed in previous SG38 meetings?

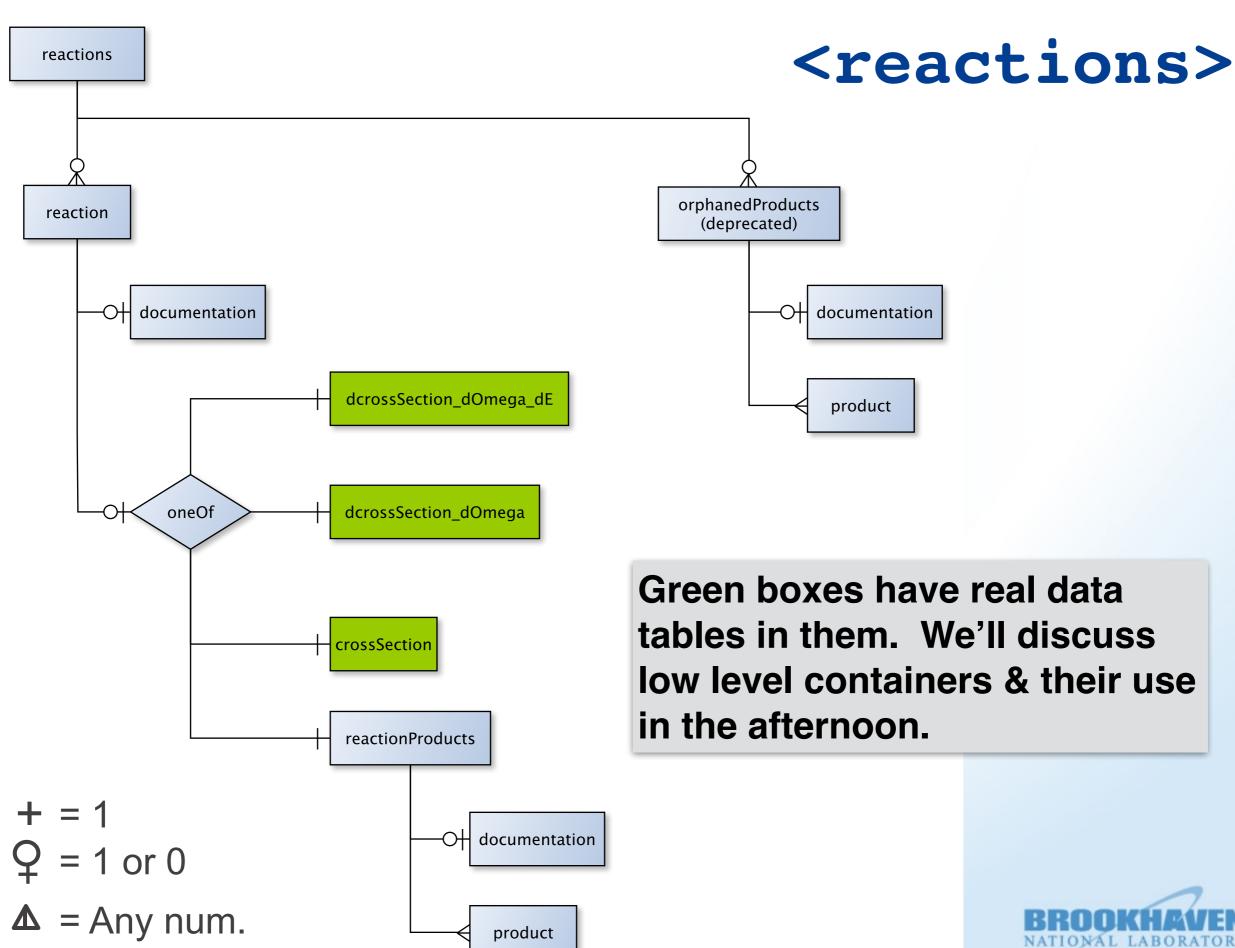
Brookhaven Science Associates

The top level: an <evaluation>, cont.

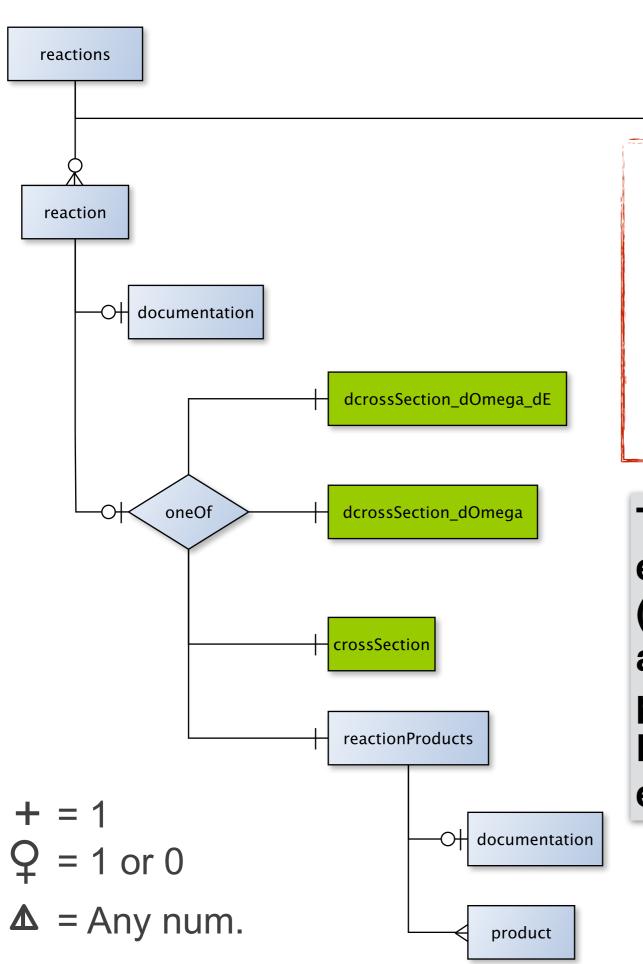


- <reactions> is the base of most kinds of reaction data
- <derivedReactions> similar to reactions, but contains reactions such as total, non-elastic, etc. which are derived from main <reactions>
- <resonances> are for anything explicitly described by the R matrix (basically ENDF MF=2, MT=151) — will discuss this afternoon

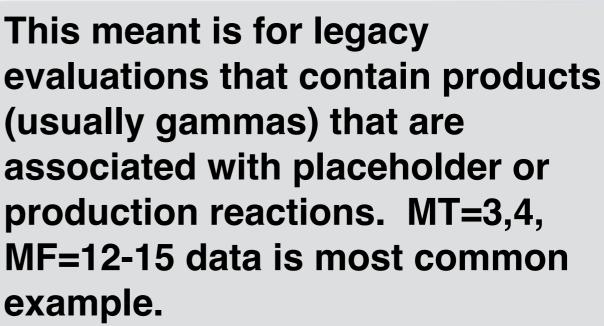
BROOKHAVEN NATIONAL LABORATORY







<reactions>



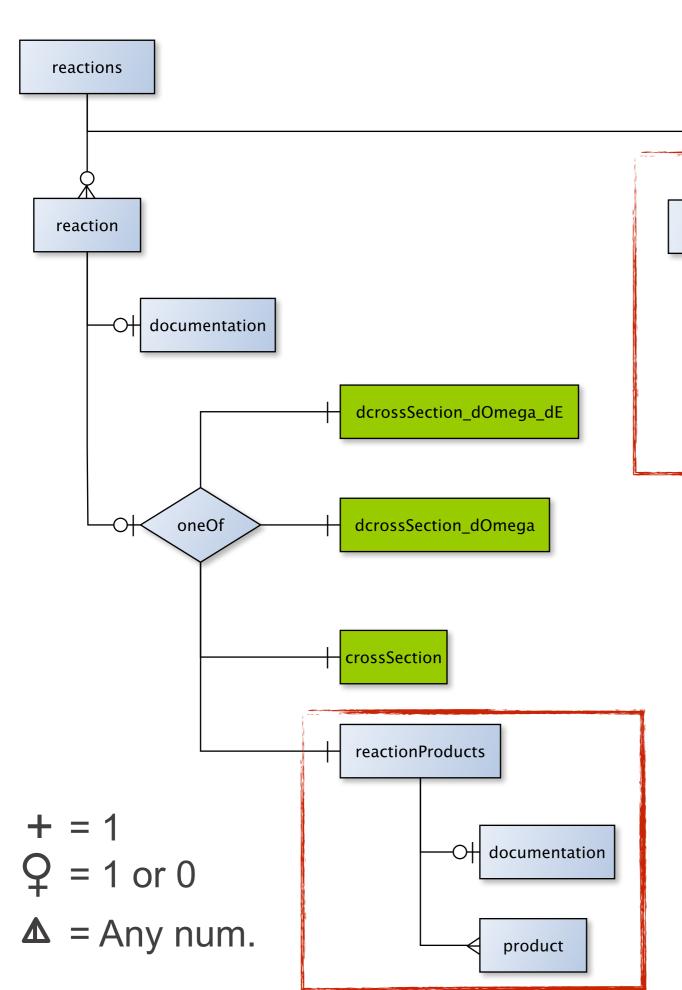
documentation

product

orphanedProducts

(deprecated)





<reactions>

This arrangement of a "list of products" is a reoccurring theme. We'll return to it after discussing cross section data.

documentation

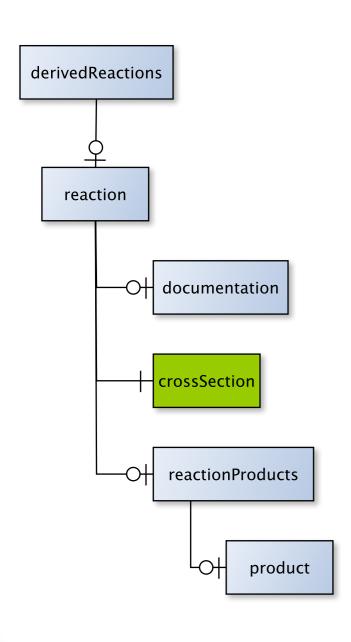
product

orphanedProducts

(deprecated)



An aside: <derivedReactions>

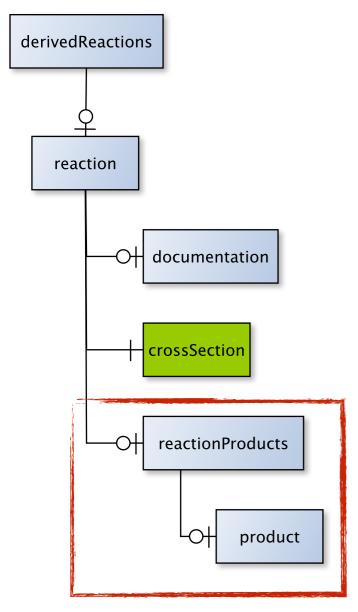


$$+ = 1$$

 $Q = 1 \text{ or } 0$
 $\Delta = \text{Any num.}$

- Typical uses:
 - (n,tot) (MT=1) cross section is sum of all other cross sections, but often well measured.
 - total fission (MT=18) cross section is sum of first, second, ... chance fission
 - Activation or particle production too
- Having the cross section here allows us to store covariance on the cross section
- Sumrule must be stored in <reaction> element
- Not meant for transport!!!
- Allows evaluator option to store total inelastic gammas instead of using <orphanedProducts> (should be deprecated)

An aside: <derivedReactions>



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- (n,tot) (MT=1) cross section is sum of all other cross sections, but often well measured.
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- Allows evaluator option to store total

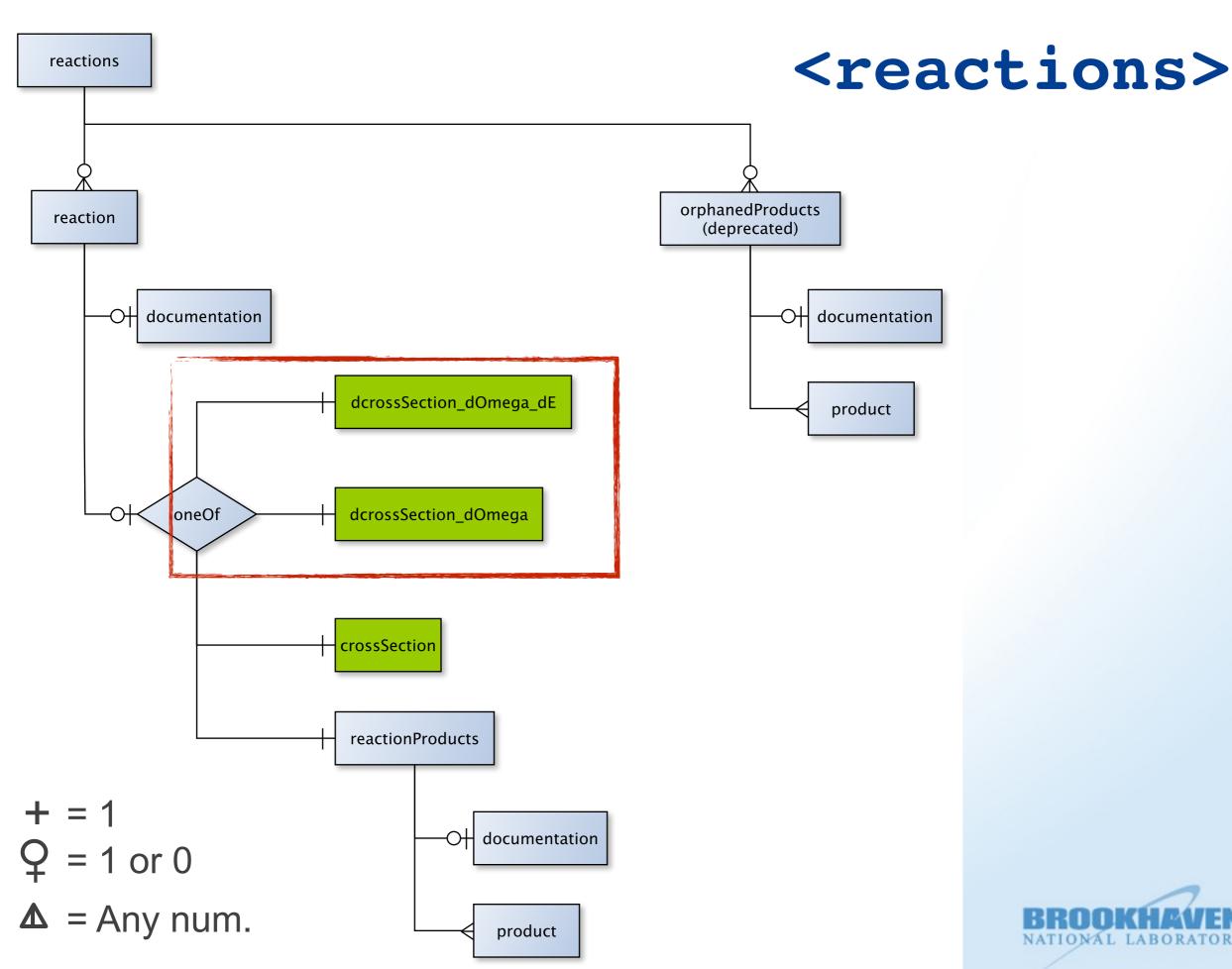
products" again

Note "list of lic gammas instead of using anedProducts> (should be cated)

13

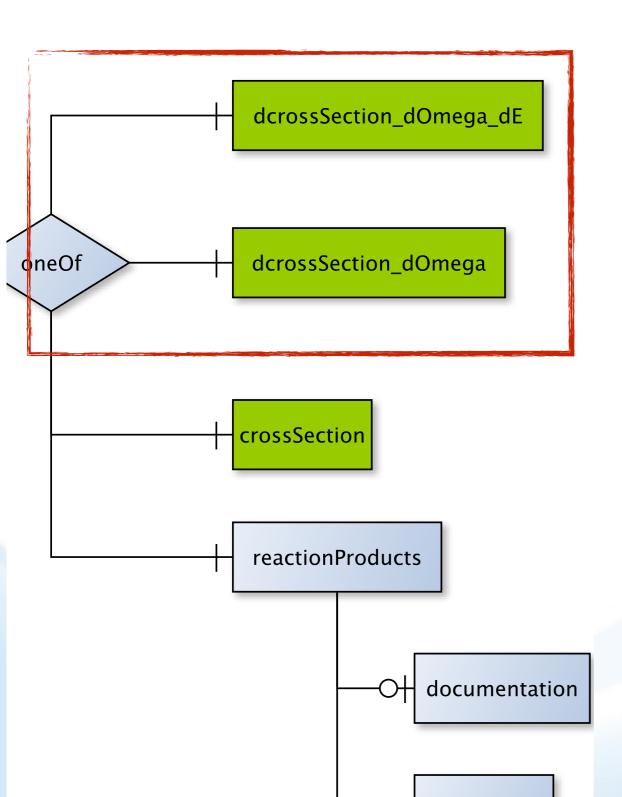
= Any num.

+ = 1



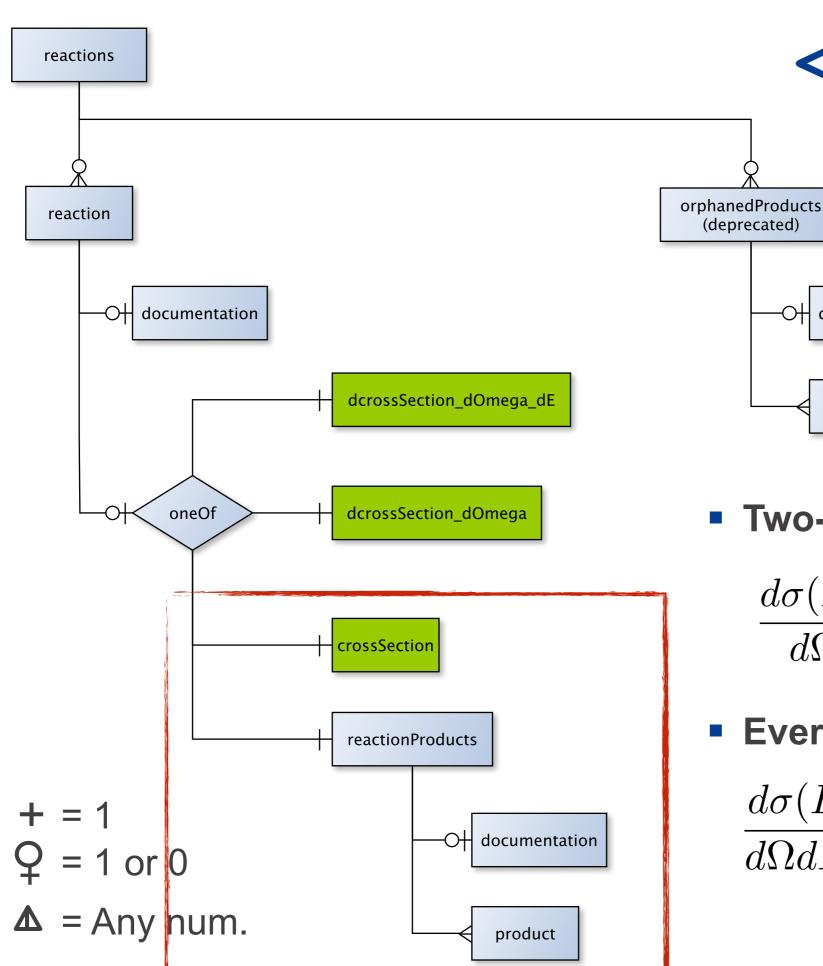


ENDF has a few special cases that require parametric or other treatment of the cross section



- With two-body kinematics, store $d\sigma(E)/d\Omega$ as parameterized data:
 - Atomic scattering using Klein-Nishina formula
 - Large Angle Coulomb Scattering (LACS) data
- Without two-body kinematics, store $d\sigma(E)/d\Omega dE'$ itself
 - Thermal Scattering Law (TSL)





<reactions>

Two-body reactions:

○H documentation

product

$$\frac{d\sigma(E)}{d\Omega} = \frac{1}{2\pi}\sigma(E)P(\mu|E)$$

Everything else:

$$\frac{d\sigma(E)}{d\Omega dE'} = \frac{1}{2\pi}\sigma(E)P(\mu, E'|E)$$



product documentation setOfDistributions → multiplicity **distribution** reactionProducts product documentation decayProducts product documentation

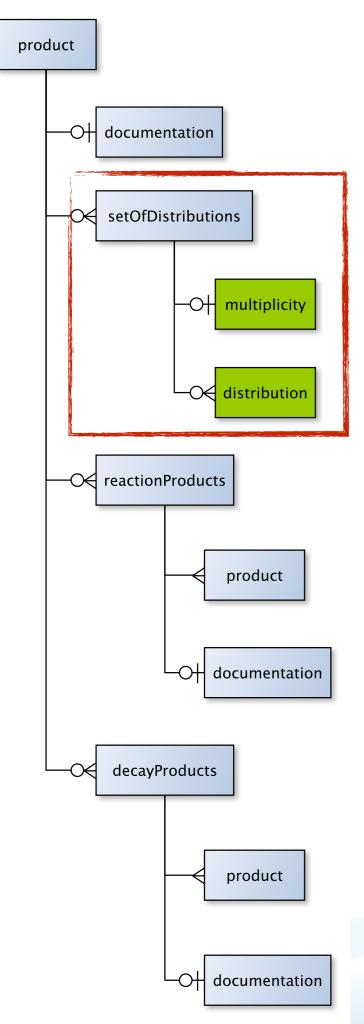


product documentation setOfDistributions -OH multiplicity Od distribution reactionProducts product documentation decavProducts product documentation

Products can break up into more reactionProducts, which have products that can breakup into more reactionProducts

Products that decay have decayProducts, decayProducts have more products which can have decayProducts which have products





Multiplicity

- may be just an int and be obvious (n,2n)
- may be interpolation table (for nubar data)
- may include P(nu|E) for fission
- possibly should be moved up a level, what do you all think?
- Distributions are divided into components; they are listed on the next slide

A <distribution > collects all the possible representations of an observable together

<pre><component></component></pre>	<form></form>	ENDF equivalent
angular	pointwise	MF4 LTT2
	Legendre	MF4 LTT1
	isotropic	MF4 LTT0
	recoil	MF6 LAW4
energy	pointwise	MF5 LF1
	Evaporation Spectrum	MF5 LF9
	Maxwellian	MF5 LF7
	Watt Spectrum	MF5 LF11
	Madland-Nix	MF5 LF12
	N-Body Phase Space	MF6 LAW6
energy-angular	Kalbach-Mann	MF6 LAW1 LANG2
angular-energy	pointwise	MF6 LAW7
uncorrelated	combination of 'angu-	MF4 and MF5
	lar' and 'energy'	